

【Track1】 Poster

SCEF Prototype for Optimizing the End-of-Life Cycle of PWR

Seung-beom Son

KEPCO Nuclear Fuel

Multi-objective Simulated Annealing based Optimization of Control Rod Design for Soluble-Boron-Free Operation

Woojin Lee

Hanyang University

Frequency-Domain Oscillation Analysis of Levitated Molten Droplets for Thermophysical Property Evaluation of Metallic Fuel Systems

Yuji Ohishi

The University of Osaka

【Track2】 Poster

Phase Diagram for Nano-Grained Oxide and Mixed Oxide Fuels

Taishi Kobayashi

Kyoto University

Mechanical Integrity and Microstructural Evolution of High Density Research Reactor Fuel Plate after High-Burnup irradiation.

Dong Jun Park

Korea Atomic Energy Research Institute

Feasibility Study on Reducing Copper Shell Thickness of KBS-3V Canisters under KJ Bentonite Swelling Pressure

Gil-Eon Jeong

Hanbat National University

【Track3】Poster

Innovative Inherent Safety Fuel Assemblies Composed of SiC Block “KAMADO-BWR & -PWR”

Tetsuo MATSUMURA

PowerM

A Review of Hydriding Mitigation of Cr coatings for Zirconium-alloy Cladding

Ganghyeon Bae

Pohang University of Science and Technology

【Track4】 Poster

Development and Verification of an Explicit OpenMC-CINDER90 Tight-Coupling Methodology for PWR Fuel Depletion: Multigroup Approximation Accuracy Budget Validated on Takahama-3

Mary Hosseini

Technical University of Munich

Comparative Study of a Creep-Based Deformation Model and BALON2 in FRAPTRAN for FFRD Analysis

Changhwan SHIN

Korea Atomic Energy Research Institute

Bayesian Machine Learning for Identifying High-Impact Fuel Material Parameters in LWRs

Donguk Kim

Seoul National University

Viscosity-based approach for 1F corium debris flow modelling using OpenFOAM

Anton P. Pshenichnikov

Japan Atomic Energy Agency

A two-stage nucleation model with incubation effect for hydride precipitation in zirconium alloys

Mikhail Kolesnik

Karlsruhe Institute of Technology

Computational Methodology for Helios-Serpent Code-to-Code Comparison of Cruciform Fuel Pin Geometry

Matilda Aberg Lindell

Lightbridge Corporation

Fuel Performance R&D for regulatory support in Korea

Yongsik Yang

Korea Atomic Energy Research Institute

POLCA8 Control Blade Depletion Benchmark against Hot Cell data

Brandon Michael Drusbasky

Westinghouse Electric Company

Strength of zirconium alloy / zirconia system interface measured by laser shock adhesion test.

Victor Asensio

CEA

Advances in Westinghouse PWR Reactor Physics Modeling Technology

Gary Mangham

Westinghouse Electric Company

Evaluation of physical properties of actinide dioxide using machine-learning molecular dynamics

Hiroki Nakamura

Japan Atomic Energy Agency

Advanced Calculation Tools to Simulate Fuel Fragment Relocation and Dispersal in Loss Of Coolant Accident Scenarios

Bruno Collard

CEA

Unified Unstructured-Mesh Framework for High-Fidelity Transport, Depletion, and Multi-Physics Coupling in Serpent 2 Monte Carlo Code

Ana Jambrina

VTT Technical Research Centre of Finland Ltd

Control Element Assembly Behavior Under Normal and Flexible Power Operation Conditions

Leo Carrilho

Westinghouse Electric Company

VALIDATION OF SRSS RESPONSE COMBINATION FOR NONLINEAR BWR FUEL ACCIDENT ANALYSIS USING MONTE CARLO TIME-PHASING

Benoit Dulauroy

framatome

【Track5】 Poster

Effects of secondary hydriding on embrittlement and fracture characteristics of Cr-coated Zr-based alloy cladding under LOCA conditions

Thi Huong Nguyen

Nuclear Safety Research Center

Analysis of Fuel Cladding Behavior during Power Ramps via EDC Tests

Jung Ju Lee

Korea Atomic Energy Research Institute

Research capabilities of the PALLAS reactor

Steven Knol

NRG PALLAS

【Track6】Poster

Mitigation of CISCC and Automated Preservice Inspection for Dry Storage Canisters: Implementation and Compliance with ASME Code Case N-860

HSIAOMING TUNG

National Atomic Research Institute

Investigation of Measures for the Stable Storage of Fuel Debris -Investigations on Degradation of Hydrogen–Oxygen Recombination Catalysts Due to Wetting, and Study on Effective Anti-Wetting Measures-

Kyoma Tsuden

Tokyo City University

Dry Cask Spent Fuel Storage at JAPC Tokai No.2 Power Station

Takumi Ogawa

The Japan Atomic Power Company

Accelerated Oxidation Study on Zircaloy-4 Cladding for CANDU Spent Fuel Dry Storage Stability

Joonho Moon

Korea Atomic Energy Research Institute

Fracture Parameters Estimation of Spent Nuclear Fuel Cladding with Reoriented Hydrides using Multi-Objective Approach

Sanghoon Lee

Keimyung University

Recent Progress of R&D on Nitride Fuels for Transmutation of Minor Actinides in JAEA

Toma Iwasa

Japan Atomic Energy Agency

【Track7】Poster

Design of a forced convection molten chloride salt loop for testing instrumentation reliability under dynamic flow conditions of advanced reactor.

Yundong Lee

Hyundai Engineering & Construction

Irradiation Performance and Qualification of High-Density Atomized U₃Si₂ Dispersion Fuel at KAERI

Tae Won Cho

Korea Atomic Energy Research Institute

Implementation of the GRSIS fission gas release model in a metal fuel performance analysis code

Cheol Min Lee

Korea Atomic Energy Research Institute

Evaluation of Nuclear Characteristics and Non-proliferation of Th-Pu Fuel in HTGRs for Plutonium Reduction

Nanami Nakao

University of Fukui

Development of NuCoT for the Prismatic-type Reactor

Tzung-Yi Lin

Radiation Protection Association

A Monte Carlo-Based Methodology for Molten Salt Reactor Fuel Cost Evaluation and Its Implications for Marine Reactor Economics

Jongwoo Lee

Hyundai Engineering & Construction