

【Track7】 Molten Salt Reactor Fuels

Development of the Molten Salt Irradiation Research Testbed (MSIRT) for In-Pile Evaluation of Fuel-Salt Performance and Structural Material Compatibility

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Expedited synthesis of UCl_3 using uranium hydride and ammonium chloride

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Fuel Performance and Hydrogen Redistribution Modelling for U-ZrH_x in a Graphite-Free AGR-like FHR

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Assessing transient performance of uranium-zirconium hydride fuel in fluoride salt-cooled reactors

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